July 14, 2022 – Session 3

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ART Advance Graphite Creep (AGC) Irradiation Experiment



Topics of discussion

- 1. Schedule
 - Effects of COVID and CIC
- 2. AGC Experiment Update
- 3. AGC-4 Status
 - Disassembly and Decon
 - Initial PIE
- 4. Anticipated areas data will be used
 - ASME code rules for irradiated graphite data
 - Support of HTR designs
 - Collaborations (Commercial vendors, NRC, GIF, IAEA)
- 5. New vendor specific irradiation capsule
 - Why? Please not another AGC experiment
 - How does it fits with new ASME code rules



AGC Irradiation Experiment: A review



Initial 600°C and 800°C irradiations

- AGC-1 and AGC-2 (600°C irradiation)
- AGC-3 and AGC-4 (800°C irradiation)
- Dose range ~ 1 to 8 dpa (for both temperatures)
- Creep data!

High Dose Graphite (HDG) capsules

- Re-irradiate previous AGC specimens
- Higher max dose (15 dpa)
- Same Temperatures (600 800°C)
- Higher dose creep data!









Irradiation material properties (AGC Experiment)



AGC-1 & AGC-2 : 600°C (0.5 to 7 dpa)

- Initial irradiation, PIE, and analysis is complete
- AGC-3: 800°C (0.5 to 3.5 dpa)
 - Initial irradiation, PIE, and analysis is complete
- AGC-4 : 800°C (3 to 8.5 dpa)
 - Irradiation complete (February 2020)
 - Specimen disassembly complete
 - We have some specimen with high rad levels
 PIE (2022 2023)
- HDG-1 : 600°C (7 to 15 dpa)
 - Back in ATR ready for irr: 2 more years to max. 15 dpa
 - Re-irradiation of AGC-2 specimens
 - Added super-fine grain sized grades => of interest for MSR designs

• HDG-2: 800°C (7 to 15 dpa)

- Irradiation begins 2023
- Re-irradiation of AGC-3 & -4 specimens to max. 15 dpa

	Pre-Irr testing	Design Capsule	Assemble & Insert	Irradiate	PIE	Analysis
AGC-1						
AGC-2						
AGC-3						
AGC-4						
HDG-1						
HDG-2						

Pertinent Irradiated Graphite Reports

ECAR-5345, As-Run Physics Analysis for the AGC-4 Experiment Irradiated in the ATR, January 2021

ECAR-5414, As-Run Thermal Analysis for the AGC-4 Experiment Irradiated in the ATR, April 2021

INL/EXT-21-63591, AGC-4 Disassembly Report, July 2021

AGC-4 Disassembly



Extraction of AGC-4 samples has been difficult

- Approximately 4-5 samples were clearly crushed
- Center stack region needed to be machined out
- But ... disassembly completed late FY21

High activity levels detected

- Appears that several specimens have high rad levels
- Special decon glovebox set-up
- Decontamination of all specimens
 - Activity levels measured for individual specimens
- Early indications indicate more than simple contamination resulting from disassembly
 - Preliminary results point to Nickel in graphite

PIE options based on activity levels

- AGC-4 PIE has begun on low rad level specimens
 - Initial shipment = 3 samples. 20+ in June 2023
 - Will be shipping piecemeal. Ship as available
- If activity levels are too high → Limited PIE on the desert
 - Want to avoid this if at all possible

Extracting piggyback samples From machined Graphite Body



Broken half of graphite body

AGC-4 PIE Status

New Split-Disk Strength



Stiffness modulus



ASTM C 769

ASTM D8982

Physical & Thermal Properties Testing

- Density
- Coefficient of Thermal Expansion
- Thermal Conductivity
- Resistivity

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- Resonant Frequency (E_{DYN})
- Torsional Frequency (G_{DYN})
 - Sonic Velocity
- Fracture Character*

*Not a non-destructive evaluation

Tested 3 specimens so far ...

Current batch of specimens being tested

CTE

Due to decontamination expect small batches

ASTM E 228-06

- Several small batches of specimens
- Much longer time to test

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Who/What will use the data?

Commercial reactor design (Direct)

- Any design using the same parameters of AGC Experiment can use the data directly
 - Same graphite grade, *T*_{Irr} range : 500 850C, dose range : 1 to 8 dpa (15 dpa after HDG)
- Irradiation dimensional change, creep rate, and material property changes

Commercial reactor design (Indirect)

- Other HTR designs can indirectly use the AGC irradiation behavior and creep data
- Combined with the ASME code methodology the data can be used to demonstrate similar behavior
 - Will need to justify how the graphite is similar

ASME code development (Steve Johns)

- Used to justify universal graphite response up to turnaround
 - Up to turnaround: All grades behave similarly
 - Past turnaround dose: Grades are not similar
 - So long as your graphite grade is within the data "cloud"
- Similar methodology for creep response/rate



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Who/What will use the data?

NRC/Licensing questions on irradiation behavior

- Training, general questions, topical reports, etc.
- Assistance with acceptance of ASME code rules

Behavior model development (Joe Bass)

- Irradiation induced stress build-up (failure determination)
- Irradiated material property changes
- Combination of degradation (no empirical data possible)
 - Irradiation + oxidation + Molten Salt

Other Collaborations

GIF, IAEA, International and National <u>Fundamental Studies</u>

Fundamental studies are designed to explain the empirically measured results



Results

Evidence of a "Buckle, ruck and tuck" defect proposed as possible underlying defect for irradiation creep







Commercial HTR irradiation capsule



Determining interest in a new graphite irradiation capsule

- DOE would provide initial capsule design for use by as many commercial vendors as possible
- DOE would assist determining MTR availability and irradiation positions (ATR and HFIR)
- DOE provides material irradiation experience

Commercial HTR designers

- Designers must determine if they require new or additional graphite irradiation data
- Designers pay for completion of capsule design, assembly, and specimen PIE
 - Specimen size, testing, irradiation dose, temperature, creep, etc.
- Capsule intended to be shared by multiple vendors cost share

Provides limited data needed for ASME's "before turnaround" rules

- This capsule will provide the dozen or so data points as explained previously
- Saves time, schedule, and cost







ADVANCED REACTOR TECHNOLOGIES

Idaho National Laboratory