

July 26, 2023

**Will Windes**

ART Graphite Technical Lead  
Idaho National Laboratory

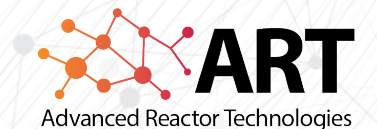
# DOE ART Graphite Program

Status 2023

**DOE ART Gas-Cooled Reactor (CR) Review Meeting**

Virtual Meeting

July 25 – 27, 2023



# Graphite Program Contributors to this discussion

Researcher	Expertise
Andrea L. Mack andrea.mack@inl.gov	ASME Code
Anne Campbell campbellaa@ornl.gov	PIE, Irradiation damage, Irradiation behavior
Arvin Cunningham arvin.cunningham@inl.gov	Oxidation, Split-disk testing
Austin C. Matthews austin.matthews@inl.gov	Material property testing, PIE, Oxidation
David T. Rohrbaugh david.rohrbaugh@inl.gov	Unirradiated and Irradiated material properties
Jose' D. Arregui-Mena arreguimenjd@ornl.gov	Microstructure, irradiation damage
Lu Cai Lu.Cai@inl.gov	Pebble Oxidation
Martin Metcalfe martin.p.metcalfe@gmail.com	HTR operations, ASME, ASTM

Researcher	Expertise
Mary Kaye Aimes marykaye.ames@inl.gov	Oxidation, Material testing
Michael E. Davenport michael.davenport@inl.gov	Irradiation experiments
Nidia C. Gallego gallegonc@ornl.gov	Molten salt technical lead, irradiation damage
Philip L. Winston philip.winston@inl.gov	Irradiation experiments
Rebecca E. Smith rebecca.smith@inl.gov	Graphite oxidation (irr. and unirr)
Steve Johns Steve.johns@inl.gov	Irradiation damage, Characterization, Split-disk
William Windes william.windes@inl.gov	Irradiation behavior, ASME
Wilna Geringer geringerjw@ornl.gov	ASME, Composites, Graphite

# Five different research areas

## Behavior models

- Predicts irradiated material properties and potential degradation issues
- Irradiation behavior for continued safe operation

## Licensing & Code

- Establishes an ASME approved code (for 1<sup>st</sup> time)
- Develops property values for initial components and irradiation induced changes

## Graphite R&D Program

Defines the safe working envelope for nuclear graphite and protection of fuel

## As-Fab'd Properties

- (Statistically) Establishes as-received material properties
- Baseline data used to determine irradiation material properties

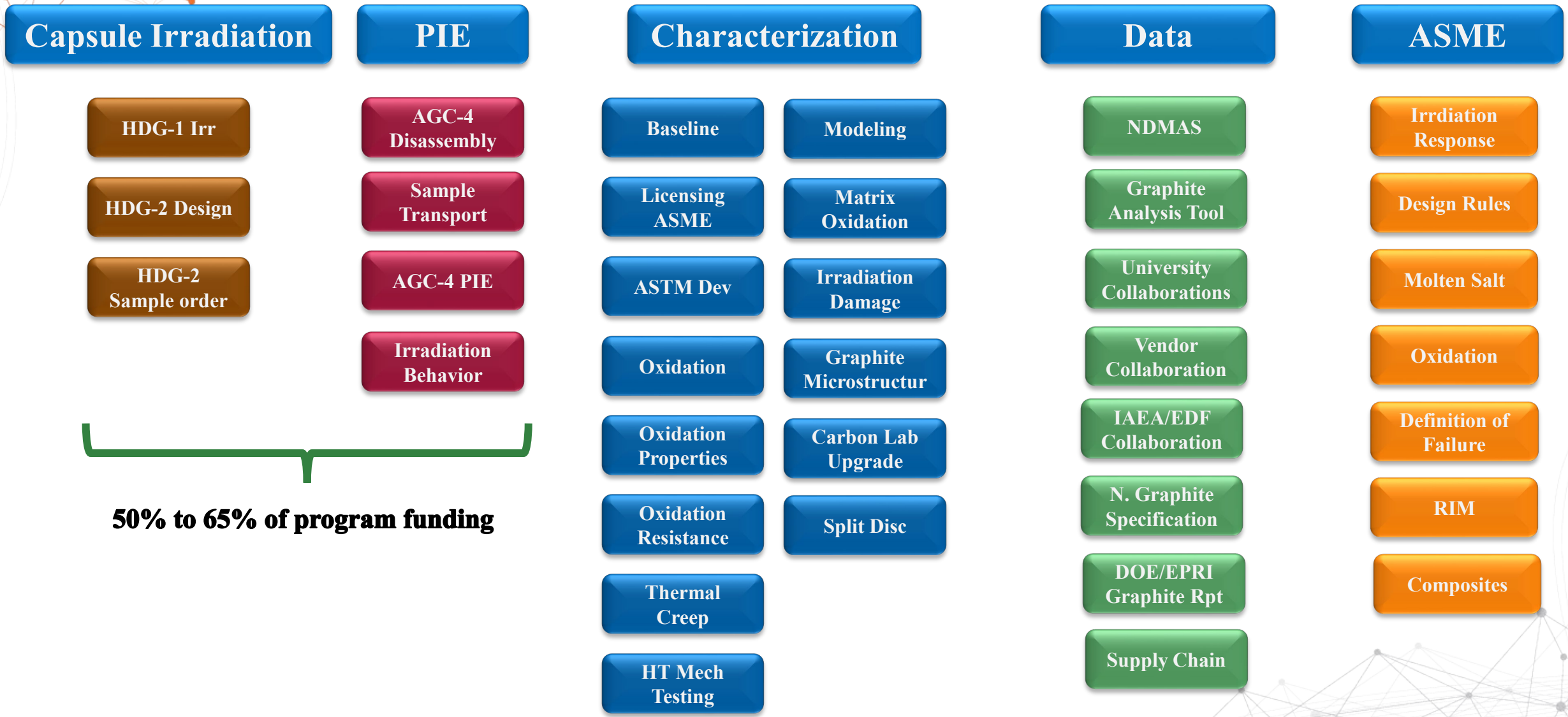
## Mechanisms and Analysis

- Data analysis and interpretation
- Understanding the damage mechanisms is key to interpreting data

## Irradiation

- Determines irradiation changes to material properties
- Irradiation behavior for continued safe operation

# FY23 Graphite Activities



# What are we discussing this year?

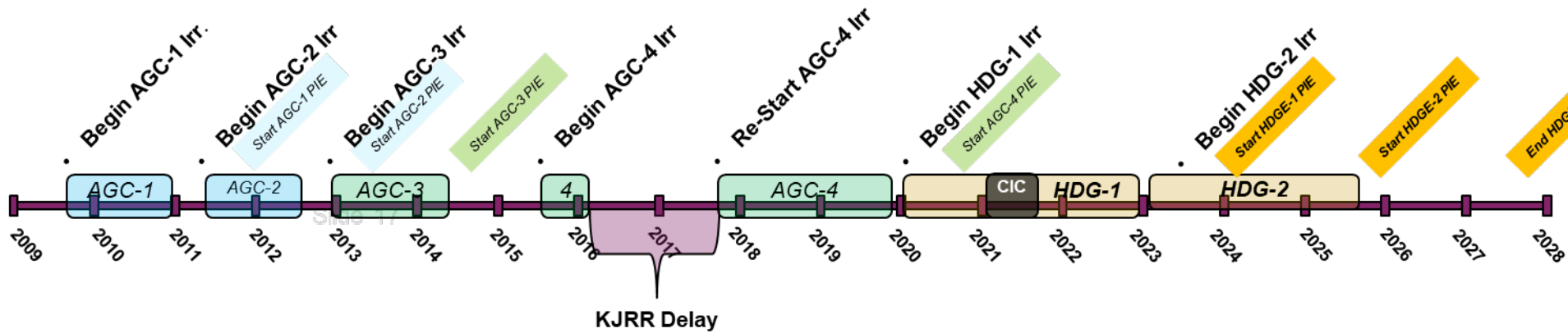
- **AGC Update**
- **Oxidation Activities**
  - *Oxidation rate, penetration/lathing, strength after oxidation*
- **Baseline Status**
  - *Data use in ASME and improved split-disc studies*
- **ASME Code Development:**
  - *Irradiation model*
  - *Ceramic Composites*
  - *Design task group*
  - *Component failure*
- **VIC Project Update**
- **Molten salt studies**
  - *We are initiating material interactions now*



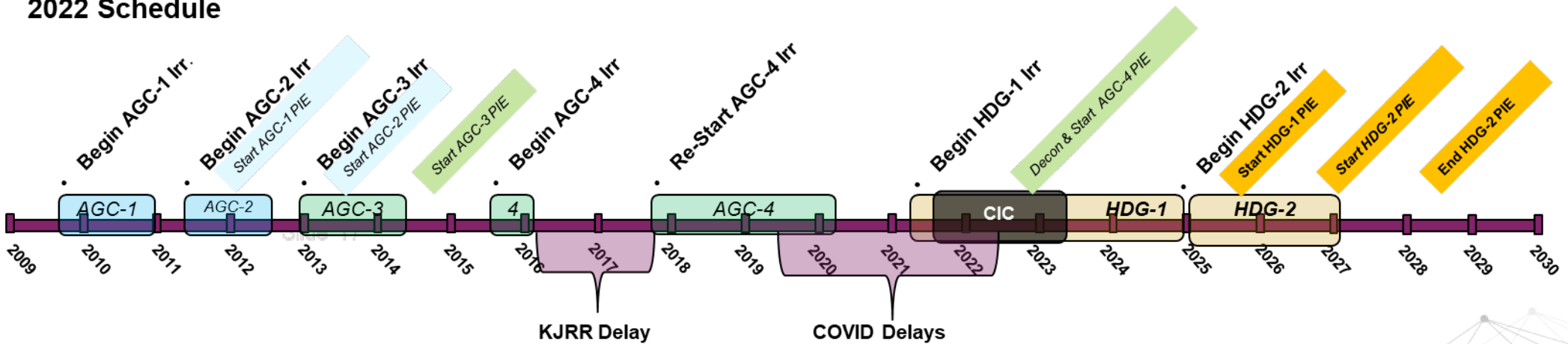
# Advanced Graphite Creep (AGC) Experiment *(Will Windes)*



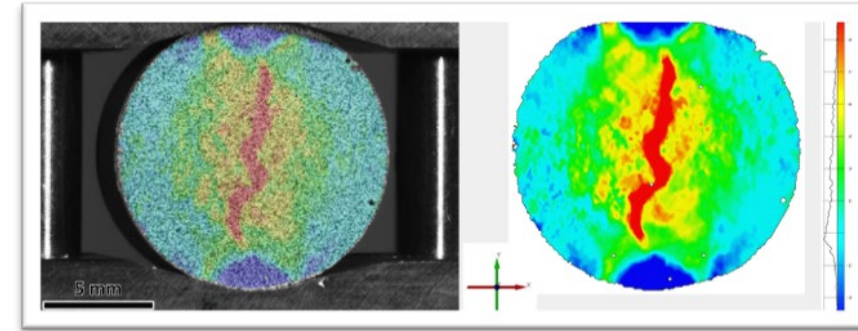
## 2018 Schedule



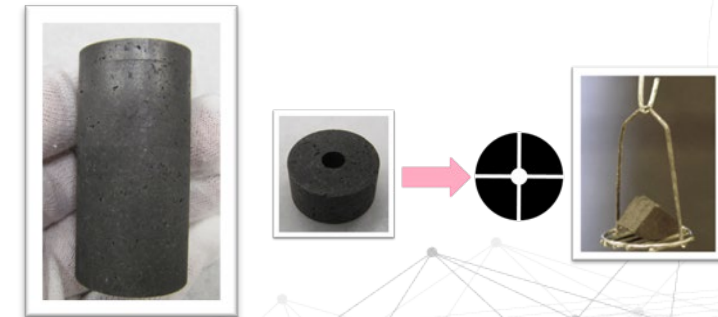
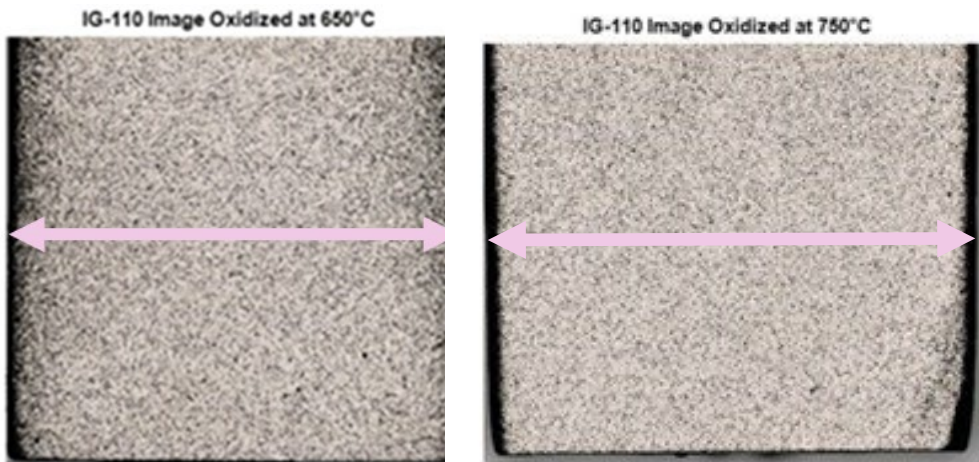
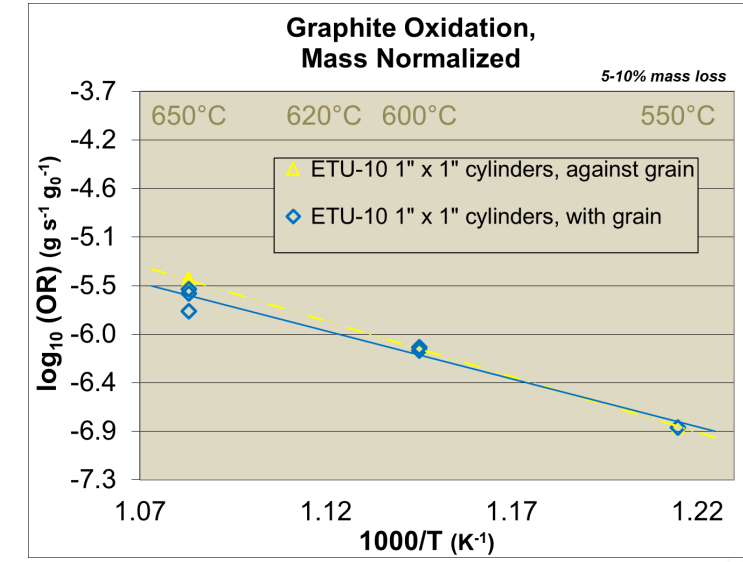
## 2022 Schedule



# Oxidation Studies *(Rebecca Smith)*



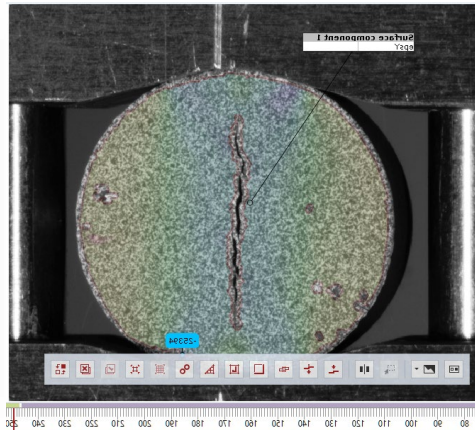
- Large amount of work being performed
  - Oxidation rate work is minimal
  - Mostly ASME and ASTM involved work
    - Strength after oxidation
    - Oxygen penetration depth
    - Oxidation of MS grades
- Oxidation of irradiated graphite



# Baseline *(Arvin Cunningham)*



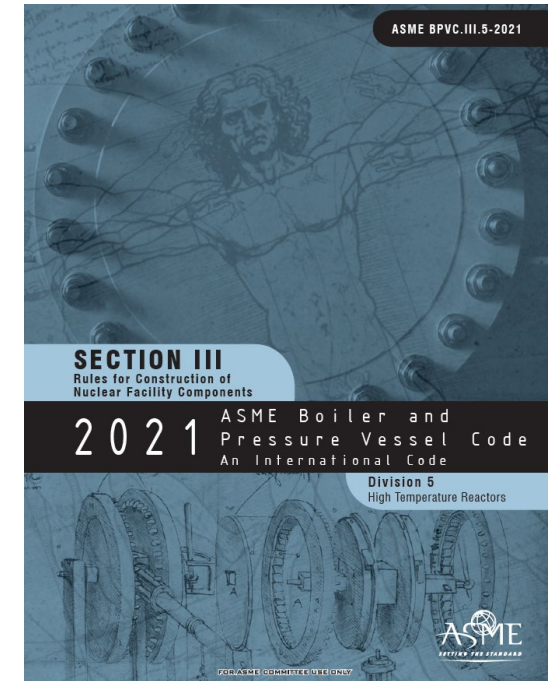
- Over 23,800 NQA-1 qualified measurements thus far
- 5 major graphite grades
- Focus is supporting other areas:
  - **Irradiation: AGC**
  - **ASTM: Split-disc testing (D8289)**
  - **ASTM: High Temperature tests**
  - **ASME code rule development**
  - **Molten salt interactions**



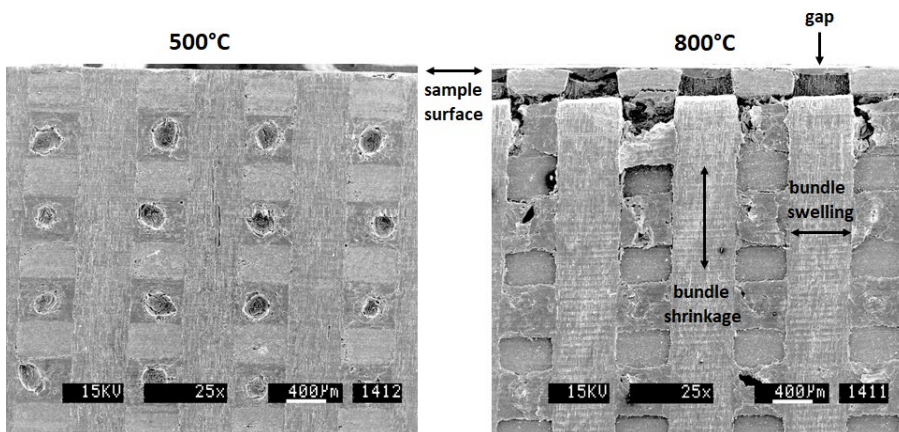
Graphite	Laboratory	Billet #	Percent Complete					Data Report	Analysis Reports	Data In NDMAS?
			Machining	Mass and Density	Elastic Testing	Mechanical Testing	Thermal Testing			
PCEA	ORNL	XPC01S8-11	100%	100%	100%	100%	100%	ORNL/TM-2015/765	ORNL/TM-2015/765	NO
PCEA	INL	XPC02S8-7	100%	100%	100%	100%	100%	ECAR-3725	INL/EXT-13-30011	YES
PCEA	INL	XPC01S8-9	100%	100%	100%	100%	100%	FCAR-6111	INL/MIS-23-70949	NO
PCEA	INL	XPC02S8-5	100%	100%	100%	100%	100%	ECAR-6110	INL/MIS-23-70951	NO
PCEA	INL	XPC01D3-35	66%							NO
PCEA	INL	XPC01D3-36	100%	100%	100%	100%	100%	FCAR-3677	INL/EXT-16-39604	YES
PCEA		Multiple Other Billets Available								
NBG-18	INL	635-4	100%	100%	100%	100%	100%	ECAR-3726	INL/EXT-14-33120, INL/EXT-13-30011	YES
NBG-18	INL	635-14	100%	100%	100%	100%	100%	ECAR-1930	INL/EXT-10-19910, INL/EXT-13-30011	YES
NBG-18	ORNL	635-6	100%	100%	100%	100%	100%	ORNL/TM-2010/219	ORNL/TM-2010/219	NO
NBG-18		Multiple Other Billets Available								
2114	INL	A20568	100%	100%	100%	100%	100%	ECAR-5798	INL/MIS-22-66680	NO
2114	INL	A20570	100%	100%	100%	100%	100%	FCAR-4322	INL/EXT-14-33120	YES
2114	ORNL	116310	100%	100%	100%	100%	100%	2018/1038, 2019/1256	ORNL/TM-2018/1038, ORNL/TM-2019/1256	YES*
2114		Multiple Other Billets Available								
NBG-17	INL	830-3	100%	100%	100%	100%	100%	FCAR-3727	INL/EXT-14-33120	YES
NBG-17	INL	V104	100%				33%			NO
IG-110	INL	089052-7	100%	100%	100%	100%	100%	ECAR-3621	INL/EXT-14-33120	YES
IG-110	INL	10X69	100%	100%	100%	100%	100%	FCAR-4182	FCAR-4182	NO



# ASME: Component Failure *(Martin Metcalfe)*



- Composite code development within ASME Section III Division 5 (HHB).
  - Approved in 2019 PBVC version
  - A nonmetallic but wholly different from graphite
    - *Highly directional (fiber orientation)*
    - *Specifically fabricated to design*
    - *Different failure behavior*
  - Initial rules require some modification



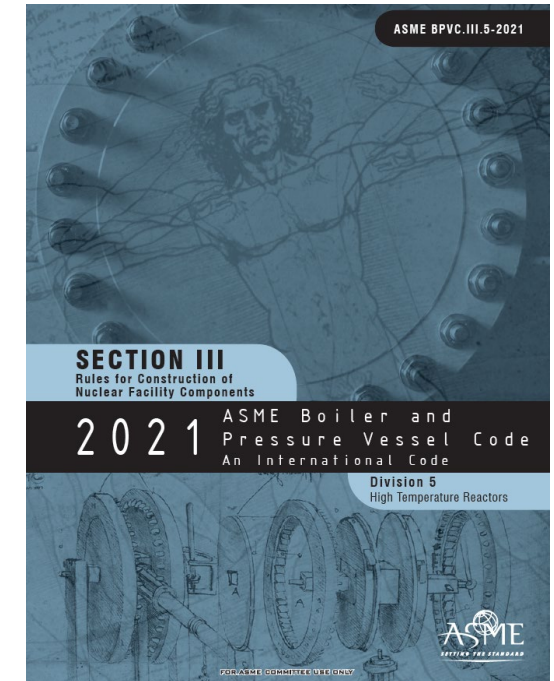
## Anticipated applications for composites in HT reactor systems.

<b>Reactor Unit</b> <b>Tie Rods</b> 	<b>Restraint Straps</b> 	<b>Interconnecting Duct</b> <b>Hot Gas Duct</b> 
<b>Control Rods</b> Example HTTR CFRC Control Rod 	<b>Power Conversion</b> <b>HP Turbine Blisk</b> 	<b>Heat Exchangers</b> 

# ASME: Design Rules *(Andrea Mack)*



- Lead Task Group in Nonmetallics Working Group (NWG)
  - *Largest commercial vendor participation*
- Priority activity in ASME Section III
  - *Sec III-5 has designated NWG Design rule changes as a priority activity*
  - *Striving to get changes into 2025 ed. BPVC*
- Heavy use of Baseline data

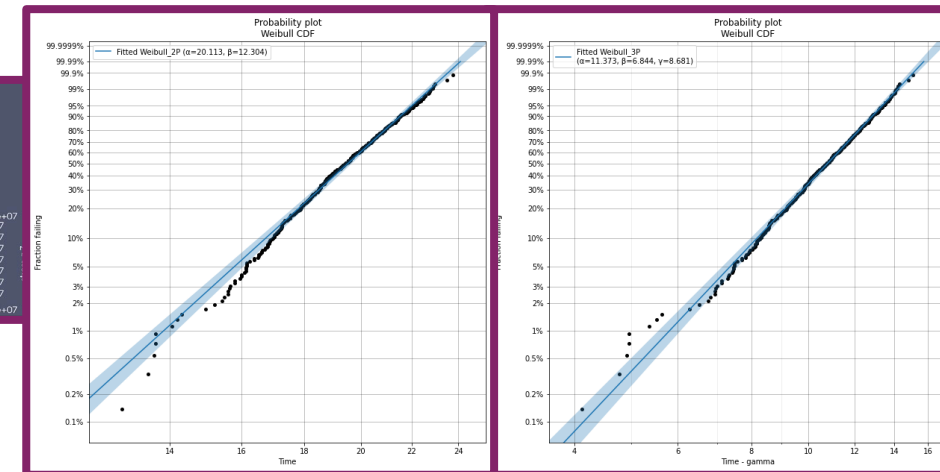
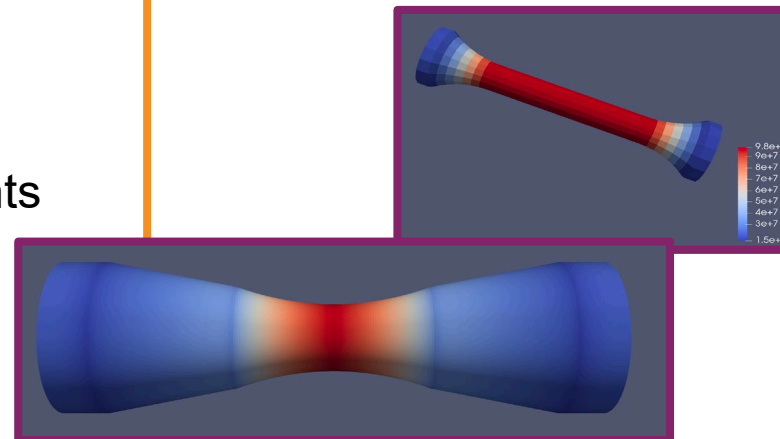


## Full Assessment

- Disparate flaw distribution
- Tuning  $V_m$  and  $\Delta$
- Mesh refinement
- Location
- Sample size requirements
- Margin

## Simplified Assessment

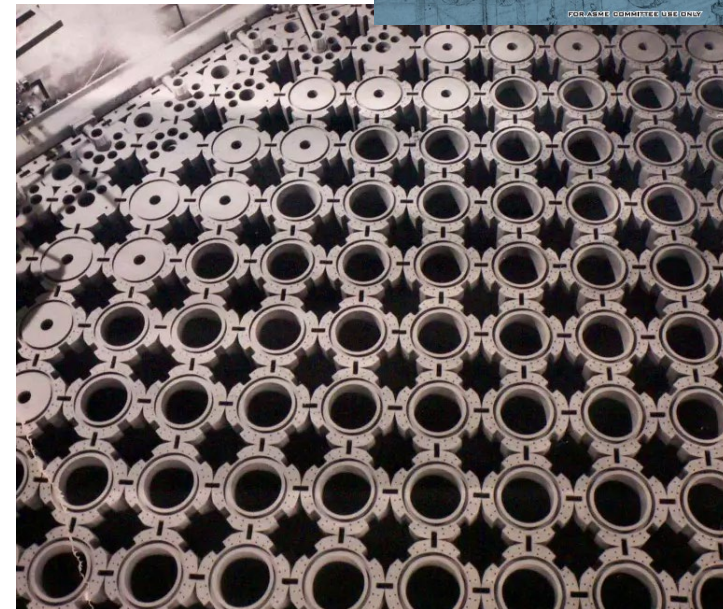
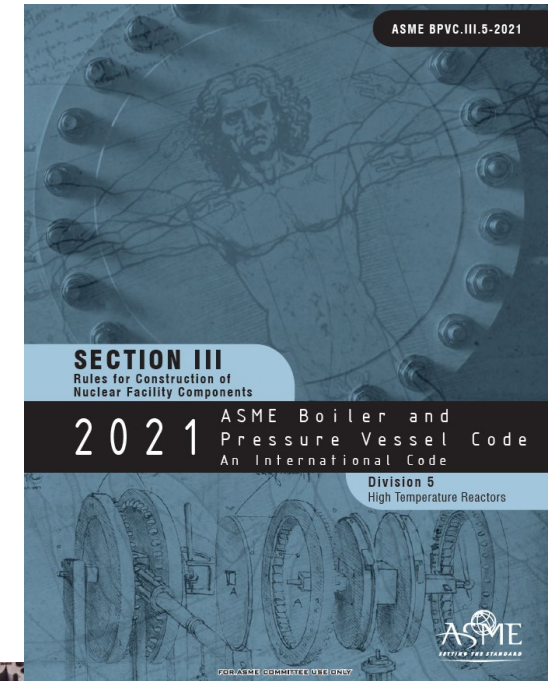
- Stress terminology
- $R_{tf}$





# ASME: Component Failure *(Martin Metcalfe)*

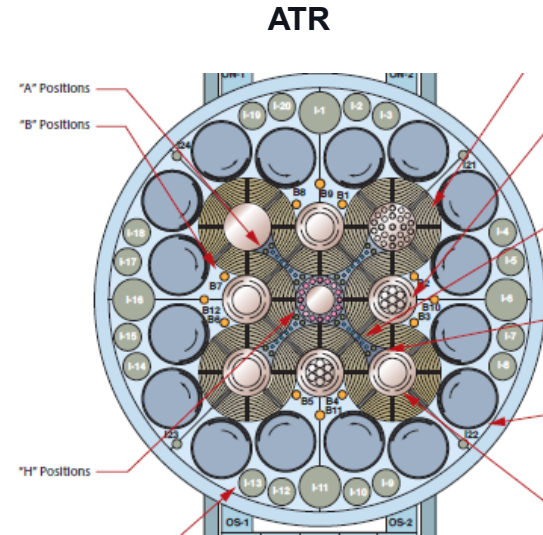
- ASME Section III Division 5: Rules for Construction of Nuclear Facility Components (High Temperature Reactors)
  - Damage tolerance and component functionality
  - Review of damage tolerance in the graphite cores of UK power reactors
  - Clarification of terminology associated with component assessment methodologies
- ASME Section XI Division 2: Rules for Inservice Inspection of Nuclear Power Plant Components (Reliability and integrity management)
  - A new supplement covering graphite components in high temperature gas reactors



# Vender Irradiation Capsule *(Will Windes)*

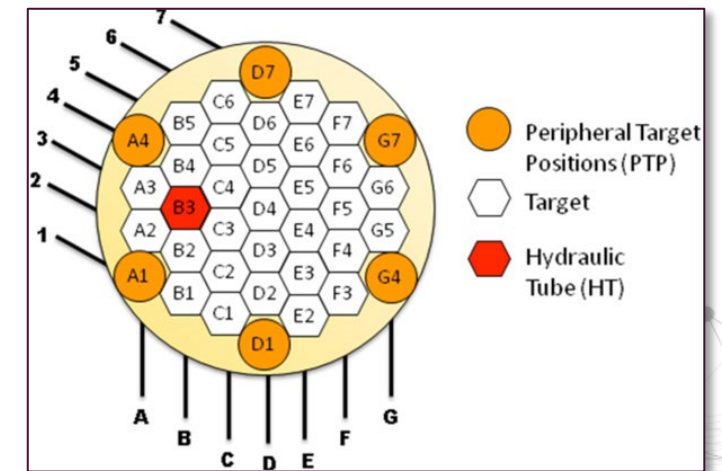


- What is the general idea
- Why is it needed?
- What are our options?
- What do the commercial vendors think?
- What is the next step



HFR (Petten)

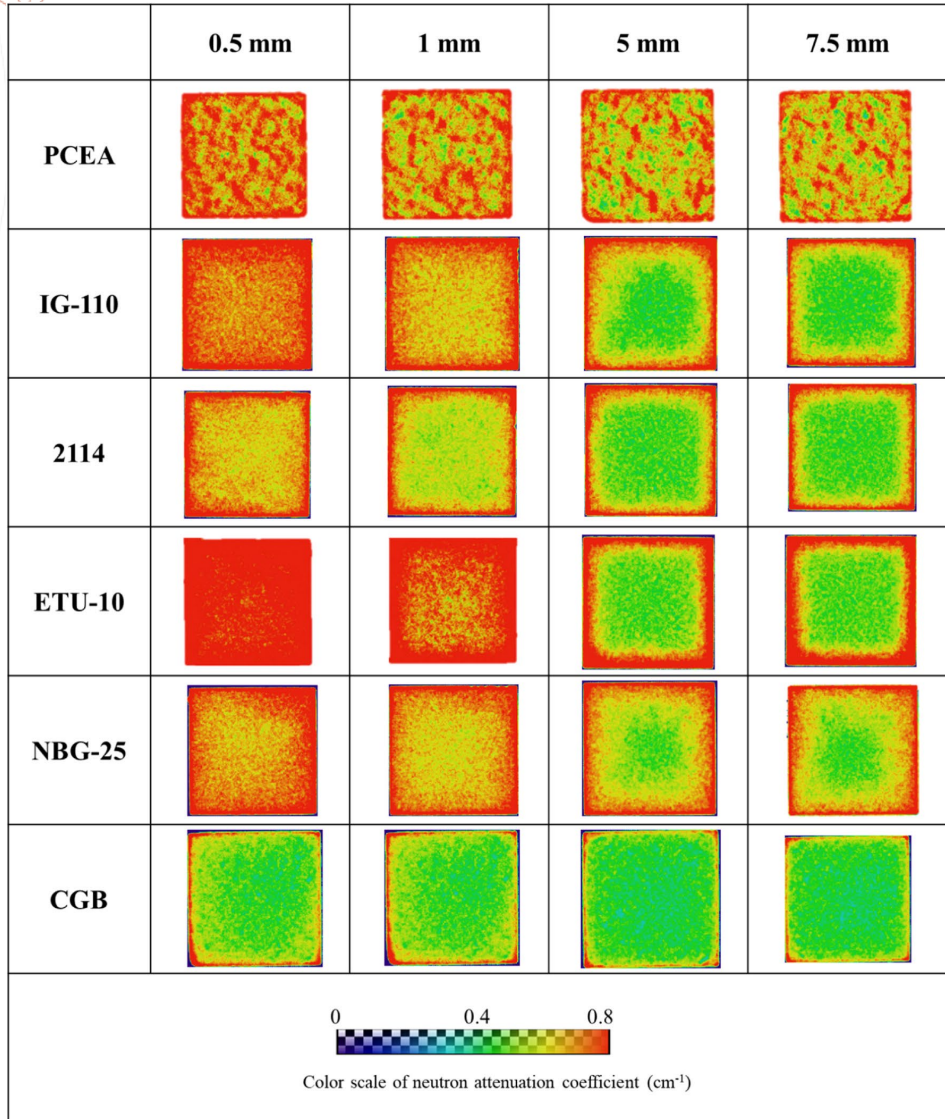
A	B	C	D	E	F	G	H	I	
+	+	+	+	1.6 0.6 0.9	+	+	+	+	1
			4.8 1.3 1.4		3.9 0.9 1.0		2.0 0.4 0.7	+	2
	8.4 2.3 1.2			6.9 1.9 1.1		4.9 1.1 0.8		+	3
							3.2 0.8 0.9	+	4
		10.7 2.8 1.5		9.2 2.5 1.4		5.8 1.5 1.0		+	5
							3.2 0.8 0.9	+	6
	8.4 2.2 1.1			7.0 1.9 1.1		4.9 1.1 0.8		+	7
			5.3 1.4 1.4		3.9 1.0 1.1		2.0 0.4 0.7	+	8
+	+	+	+	1.6 0.6 1.1	+	+	+	+	9



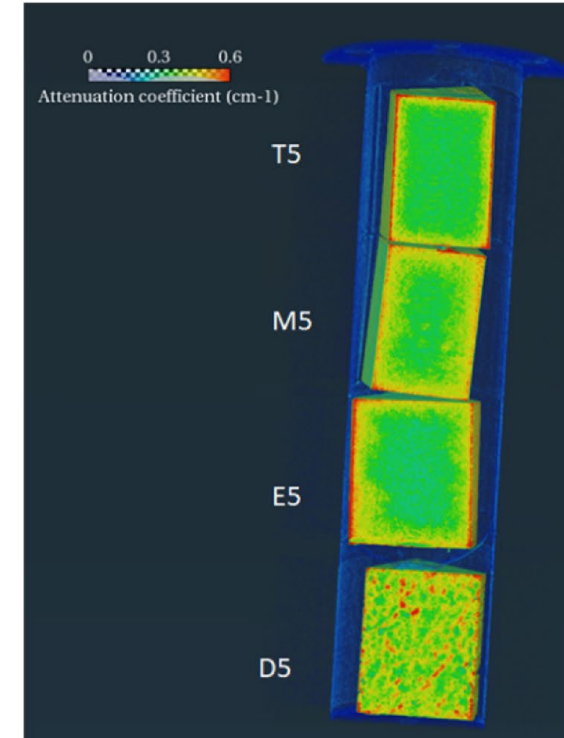




# Molten Salt – Graphite Interactions *(Nidia Gallego)*



- Molten salt is just another degradation mechanism for nuclear graphite
  - *Unirradiated and irradiated material behavior still occur*
- A status on initial molten-salt interaction studies
  - *MS penetration into graphite microstructure*
  - *Erosion/Abrasion*
- New funding molten salt – graphite interaction studies (no results yet)
  - *Enhance and complete results from these initial studies*



# NEUP: New graphite research (Thursday, July 27th, 2023)

## Advanced Reactor Component Materials Topics for FY21 NEUP Call

- RC X-Y. Effects of Irradiation Induced Microstructure Change in Graphite
  
- 1. Multiscale Effects of Irradiation Damage on Nuclear Graphite Properties
  - *Aman Haque, Pennsylvania State University*
  
- 2. Quantifying the Dynamic and Static Porosity/Microstructure Characteristics of Irradiated Graphite through Multi-technique Experiments and Mesoscale Modeling
  - *Jacob Eapen, North Carolina State University*